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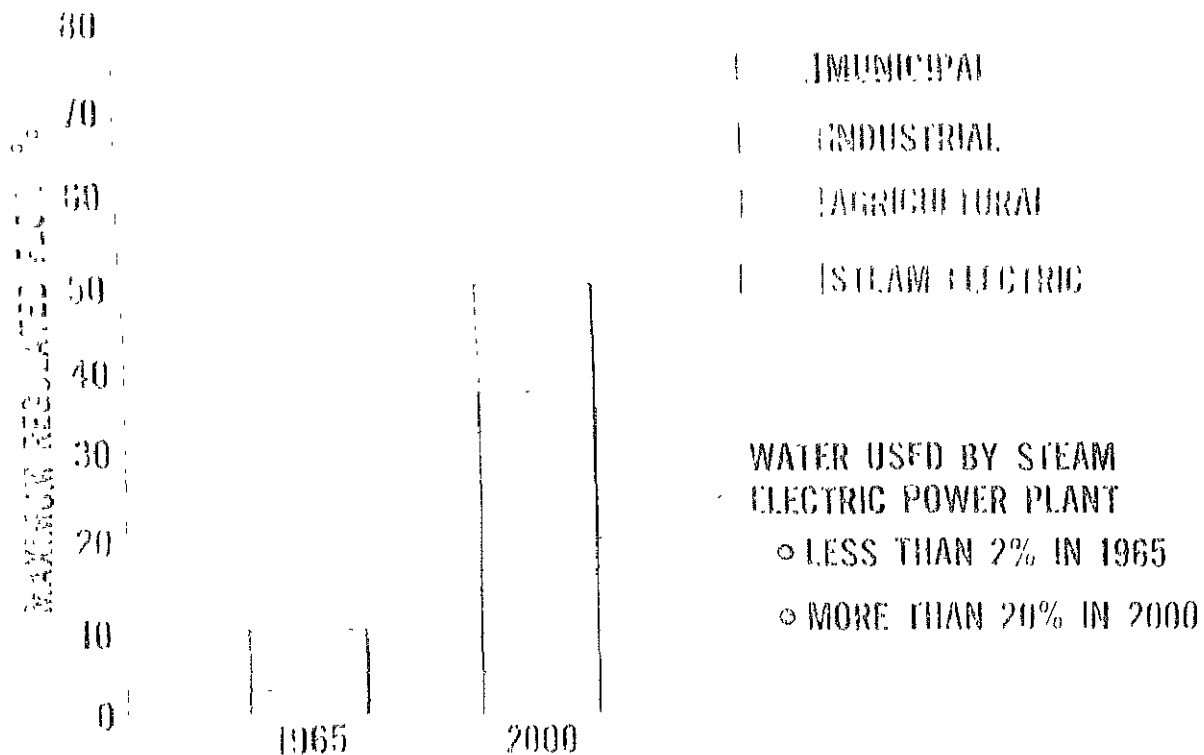
COST ACCOUNT	TITLE	MANAGER	FY-81 FUNDING	PAGE
<u>AE-05-05</u>	<u>ADVANCED TECHNOLOGY AND SYSTEMS ASSESSMENT</u>			
AKB	WATER USE INFORMATION SYSTEM	JF FLETCHER	\$ 100K	2
<u>AF-10-25</u>	<u>GAS COOLED FAST BREEDER REACTOR DEVELOPMENT</u>			
AKS	GCFR STRUCTURAL MATERIALS	LID BLACKBURN	\$ 20K	4
<u>AG-40-10-05</u>	<u>LWR FUEL CYCLE</u>			
ALA	HYBRID FUEL FABRICATION	DW BENNETT	\$ 750K	6
<u>AR-05-10-02</u>	<u>DEFENSE WASTE MANAGEMENT</u>			
AKC	ACID DIGESTION (LOW LEVEL)	CR ALLEN	\$ 1500K	8
<u>AT-15-30-31</u>	<u>FUSION REACTOR MATERIALS</u>			
AKH	FUSION ALLOY DEVELOPMENT	GL WIRE	\$ 750K	10
<u>AT-15-30-33</u>	<u>FUSION REACTOR MATERIALS</u>			
ALI	SOLID LITHIUM BREEDER DEVELOPMENT	EI WEBER	\$ 200K	12
<u>AT-15-30-34</u>	<u>FUSION REACTOR MATERIALS</u>			
AKJ	IRRADIATION EFFECTS ANALYSIS	DG DORAN	\$ 700K	14
<u>HA-01-03-04</u>	<u>REGIONAL ASSESSMENT</u>			
ALR	OTI WATER RESOURCE STUDIES	JF FLETCHER	\$ 25K	16
<u>SPECIAL REQUESTS</u>	<u>CLINCH RIVER BREEDER REACTOR PROJECT</u>			
EBA	CRBRP REACTOR SYSTEMS THERMAL HYDRAULIC TESTING	WL THORNE	\$ 890K	18
LBC	CRBRP FUEL FAILURE MONITORING	JJ McCOWN		20
		SUBTOTAL	\$ 980K	
<u>SPECIAL REQUESTS</u>	<u>NUCLEAR REGULATORY COMMISSION</u>			
EAA	SHIPPING CASK ANALYSIS	JF FLETCHER	\$ 120K	22
EAB	LWR NEUTRON DOSIMETRY	WN McELROY	588	24
EAD	LOFT ADVANCED FUEL ROD INSTRUMENTATION DEVELOPMENT	EM SHEEN	164	26
		SUBTOTAL	\$ 872K	

<u>ST</u> <u>UNT</u>	<u>TITLE</u>	<u>MANAGER</u>	<u>FY-81</u> <u>FUNDING</u>	<u>PAGE</u>
<u>TS</u>	<u>OTHLR</u>			
	NATIONAL WASTE TERMINAL STORAGE PROGRAM	RJ CASH	\$ 924K	28
	AND SPENT FUEL ENGINEERING			
	FUSION REACTOR SAFETY SUPPORT STUDIES	ED MOHLSTEIN	411	30
		SUBTOTAL	<u>\$1365K</u>	
		TOTAL	<u><u>\$7262K</u></u>	

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<u>VC-2: REACTOR CORE SUPPLY - IT NAGAMOTO</u>				
ALA	HYBRID FUEL FABRICATION	DW BENNETT	\$ 750K	6
LAD	LOFT ADVANCED FUEL ROD INSTRUMENTATION DEVELOPMENT	FM SHEEN	164	26
		SUBTOTAL	\$ 914K	
<u>VC-3: CORE EVALUATION - CM COX</u>				
ALT	SOLID TRITIUM BREEDER DEVELOPMENT	LT WEBER	\$ 200K	12
<u>VC-4: TECHNOLOGY - HH YOSHIKAWA</u>				
AKB	WATER USE INFORMATION SYSTEM	JE FELICHER	\$ 100K	2
AKC	ACID DIGESTION (LOW LEVEL)	CR ALLEN	1500	5
AKH	FUSION ALLOY DEVELOPMENT	GL WIRE	750	10
AKJ	IRRADIATION EFFECTS ANALYSIS	DO DORAN	700	14
AKS	GAS COOLED FAST REACTOR STRUCTURAL MATERIALS	ED BLACKBURN	20	4
ALR	OFF-WATER RESOURCE STUDIES	JE FELICHER	25	16
EAA	SHIPPING CASK ANALYSIS	JE FELICHER	120	22
EAB	LWR NEUTRON DOSIMETRY	WN McFEROY	588	24
EBA	CRBRP REACTOR SYSTEMS THERMAL-HYDRAULIC TESTING	WT THORNIL	890	18
EBC	CRBRP FUEL FAILURE MONITORING	JJ McCOWN	90	20
LCA	NATIONAL WASTE TERMINAL STORAGE PROGRAM AND SPENT FUEL ENGINEERING	RJ CASEY	924	28
LCB	FUSION REACTOR SAFETY SUPPORT STUDIES	ED MUEHLSTEIN	441	30
		SUBTOTAL	\$ 6148K	
		TOTAL	\$ 7262K	

WATER RESOURCE EVALUATIONS



WATER USE INFORMATION SYSTEM

- INDIVIDUAL POWER PLANT CHARACTERISTICS
- REGIONAL HYDRAULIC DATA
- WATER RIGHTS
- REGIONAL LOAD FORECASTS
- REGIONAL WATER USAGE



WATER USE INFORMATION SYSTEM

HEDL

Cost Account - AKB

Oper. X Inv.

Manager: J F. Fletcher
Department: Technology

Cost: \$ 100K

Customer:
Organization: DOE/ANSP
Contact: W F. Savage
Program: Advanced Technology and
Systems Assessment
AE-05-05

OBJECTIVE

Provide data and evaluations on water resource availability in support of the advanced technology systems program

PE

Develop and operate the computerized Water Use Information System. Perform national and regional water resource evaluations at DOE request

RECENT TECHNICAL HIGHLIGHTS

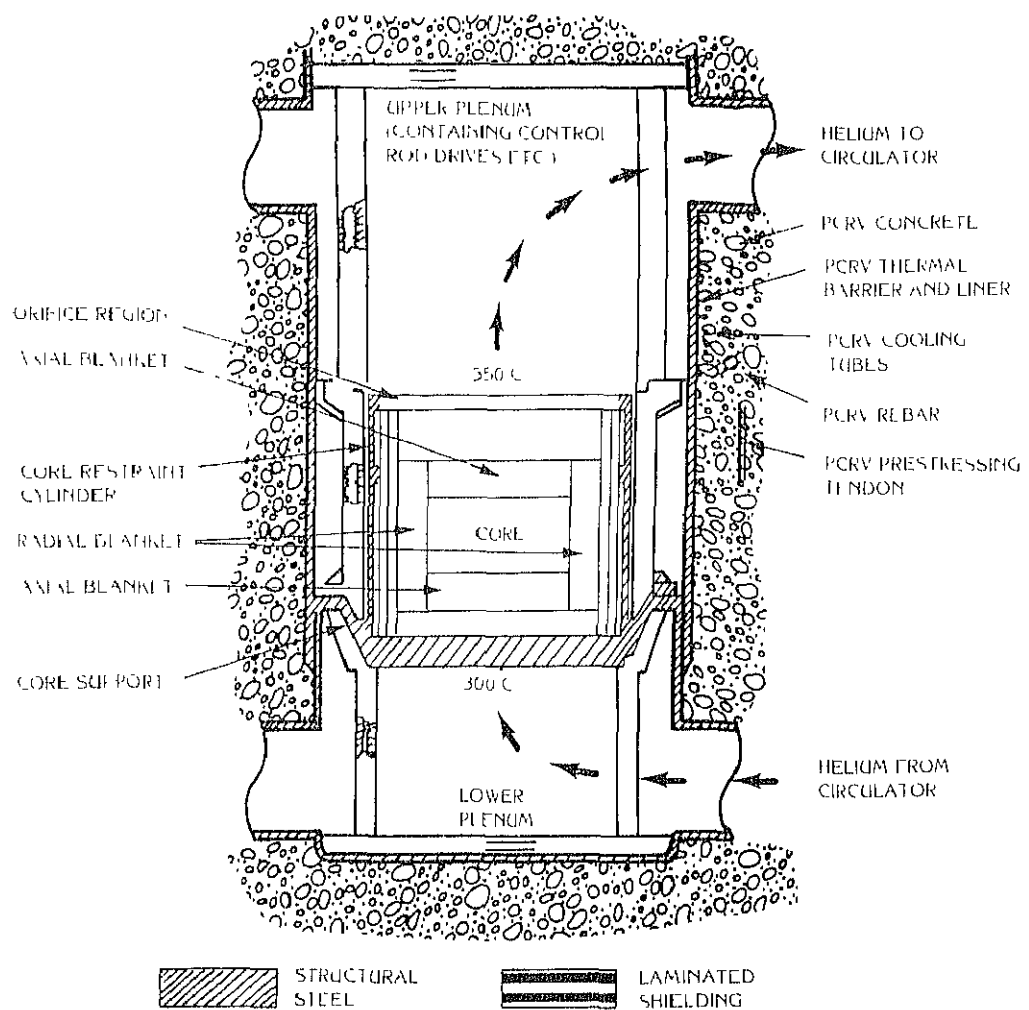
1. An econometric predictive model for electricity demand, was completed and used in national water resource evaluations.

2. Preliminary water resources evaluation of national and regional water availability for power plant cooling was completed.

PROJECTED NEAR-TERM ACCOMPLISHMENTS

1. Updated, detailed study of national and regional water availability for power plant cooling to be completed.

CROSS SECTION OF GCFR DEMONSTRATION PLANT REACTOR CAVITY



GAS COOLED FAST REACTOR STRUCTURAL MATERIALS

WEDL
Cost Account - AKS
Oper. X Inv.

Manager: L D. Blackburn
Sub-Department: Technology

FY-81: \$ 20K

Customer:
Organization: DOE/RRT
Contact: G.A. Newby
Program: Gas Cooled Fast
Breeder Reactor
Development
AF-10-25

OBJECTIVE

Provide mechanical property data on irradiated and unirradiated materials to support the design, safety analyses, and operation of GCFR out-of-core components.

SCOPE

Conduct irradiations and testing to determine strength, ductility, fatigue crack propagation and fracture toughness of selected materials.

RECENT TECHNICAL HIGHLIGHTS

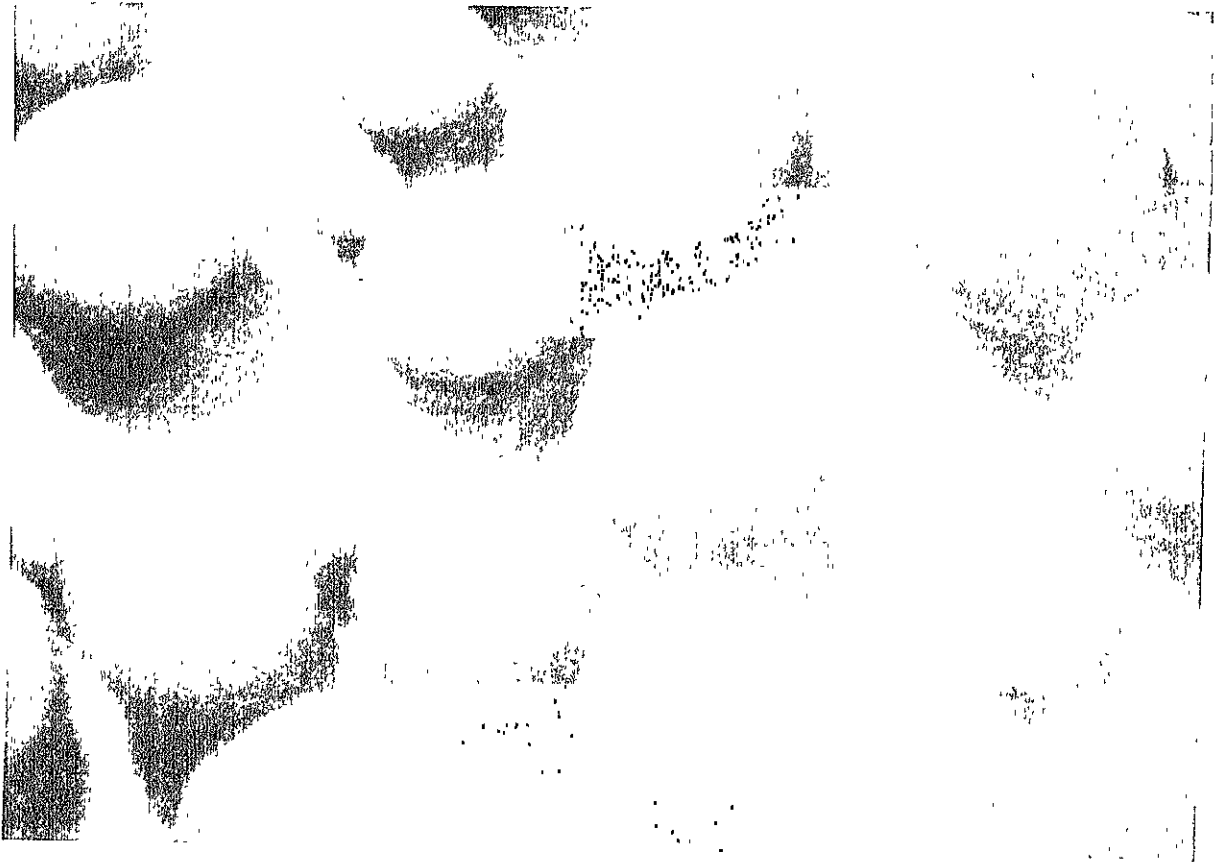
EBR-II irradiation of three pins and postirradiation tension testing to determine high temperature fluence limits for 316 SS materials was completed.

Characterization of fatigue crack propagation in two ferritic steels, modified 9Cr-1Mo and HT-9, was completed.

EXPECTED NEAR-TERM ACCOMPLISHMENTS

GCFR Program is closing out. Documentation of tests and evaluation will be completed in FY-81.

GEL SPHERES



UNSINTERED ~40X

HYBRID FUEL FABRICATION

HEDL

Cost Account - ALA

Oper. _____ Inv. _____

Manager: D.W. Bennett

Sub-Department: Reactor Core Supply

FY-81: \$ 750K

Customer:

Organization: DOE/NPD

Contact: W.W. Ballard

Program: LWR Fuel Cycle
AG-40-10-05

OBJECTIVE

Evaluate the equipment required for fabrication of breeder reactor pellet fuels from an alternate conversion source.

SCOPE

Adapt equipment used to fabricate fuel pellets from mixed oxide powder to make fuel from gel-spheres produced by the internal gelation method.

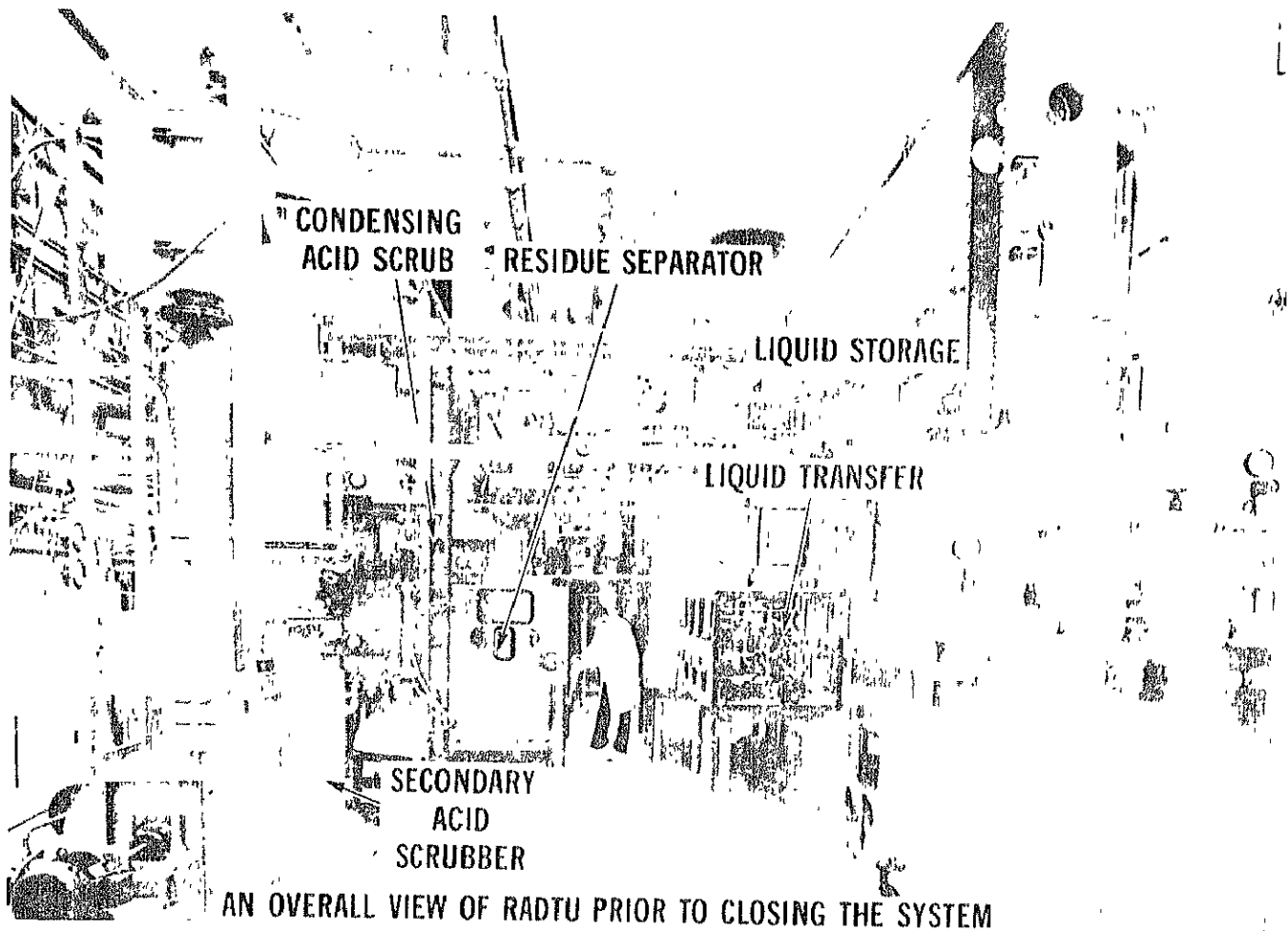
Adapt as necessary automated analytical chemistry techniques, waste/scrap processes, and handling systems to accommodate gel/sphere processing.

RECENT TECHNICAL HIGHLIGHTS

The first sample of gel-spheres was received and is being evaluated and analyzed.
Press feeding system was designed and fabricated.

EXPECTED NEAR-TERM ACCOMPLISHMENTS

Transporting and feeding tests will be conducted with spheres.
Press feed system will be demonstrated.
Cost comparison data will be established.



ACID DIGESTION
(LOW LEVEL)

HEDL
Cost Account - AKC
Oper. X Inv.

Manager: C R Allen
Sub-Department: Technology

FY-81: \$ 1500K

Customer:
Organization: DOE/RL
Contact: G Miskho/W C. Johnson
Program: Defense Waste Management
AR-05-10-02-F

OBJECTIVE

Process Z-plant transuranic waste and scrap. Demonstrate capability of acid digestion system to process various waste and scrap forms.

SCOPE

Demonstrate capability of the Radioactive Acid Digestion Test Unit (RADTU) to process Z-plant waste, D&D waste, and special waste forms.

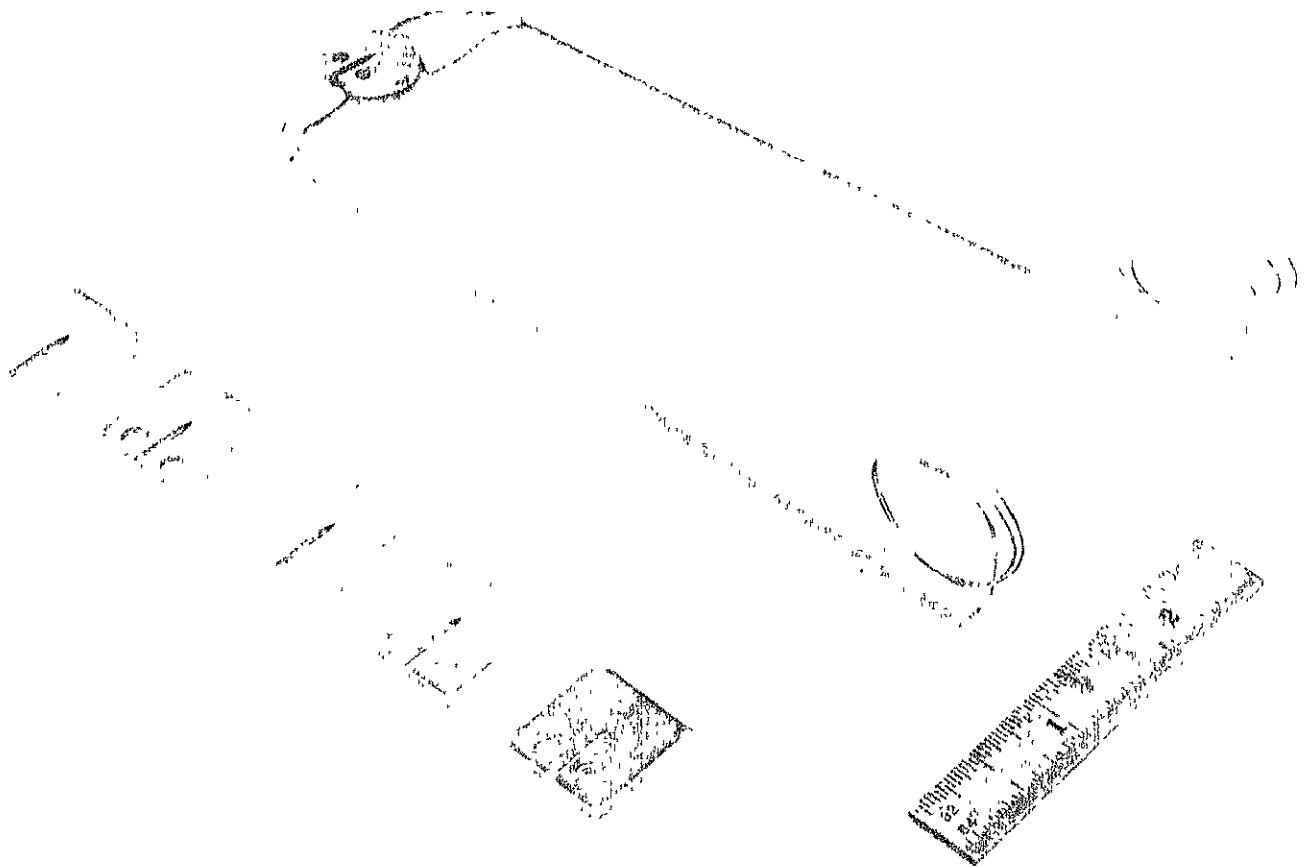
RECENT TECHNICAL HIGHLIGHTS

High rate digester equipment was installed and tested to increase system capacity to 10 kg waste/hr. Centrifuge and air classification equipment was installed and tested. Radioactive operation was restarted in June, 1980.

International workshop on acid digestion was held at HEDL in October, 1980.

EXPECTED NEAR-TERM ACCOMPLISHMENTS

Demonstration of high activity waste processing will be completed in CY 1980. Tests in RADTU to determine processability of "unique solutions" and reactive scrap forms will be completed in FY-81.



MAGNETIC FUSION ENERGY-5 (MFE-5) IN
REACTOR FATIGUE CRACK GROWTH EXPERIMENT

FUSION ALLOY DEVELOPMENT

HEDL
Cost Account - AKH
Oper. X Inv.

Manager: G.L. Wile
Sub-Department: Technology

FY-81: \$ 750K

Customer:
Organization: DOE/OFE
Contact: T.C. Reuther
Program: Fusion Reactor Materials
AT-15 30 31

OBJECTIVE

Develop, characterize and qualify materials for fusion reactor wall applications.

SCOPE

Test and analyze irradiated materials to determine fatigue, fracture toughness, swelling, and creep.

RECENT TECHNICAL HIGHLIGHTS

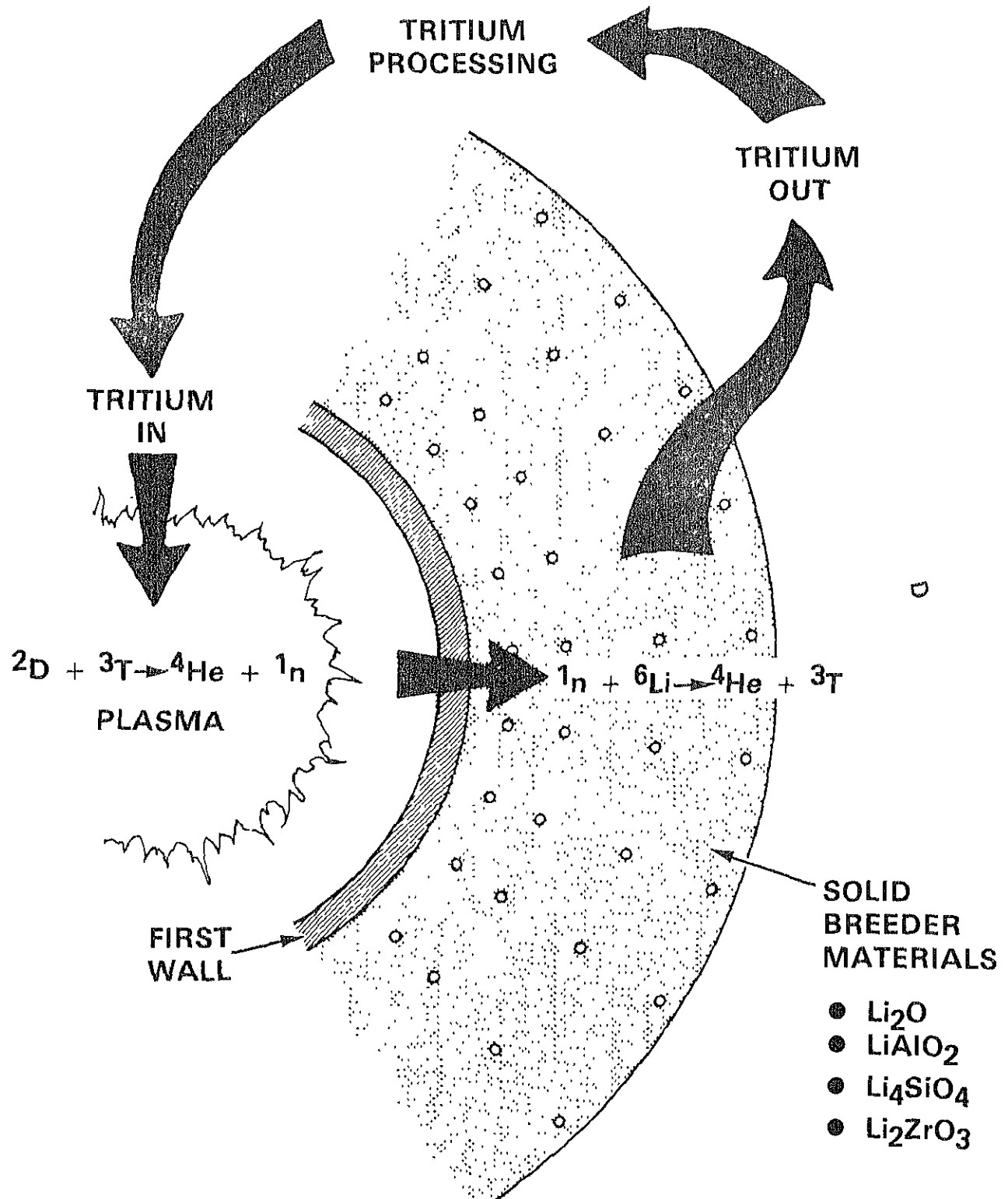
Fabrication of the world's first MFE-5 in-reactor fatigue and crack growth experiment was completed and the test assembly is ready for insertion into the Oak Ridge Research Reactor.

EXPECTED NEAR-TERM ACCOMPLISHMENTS

Irradiation of the MFE-5 experiment in the Oak Ridge Research Reactor will be started. Post irradiation fatigue crack growth tests on titanium will be started.

SOLID BREEDER MATERIALS IN FUSION REACTORS

● A BASIC ELEMENT OF THE TRITIUM CYCLE



SOLID TRITIUM
BREEDER DEVELOPMENT

HEDL
Cost Account - ALT
Oper. X Inv.

Manager: E.T. Weber
Sub-Department: Core Evaluation

FY-81: \$ 200K

Customer:
Organization: DOE/OFE
Contact: T.C. Reuther
Program: Fusion Reactor Materials
AT-15-30-33

OBJECTIVE

Develop, characterize and qualify blanket materials for fusion reactor applications

SCOPE

Fabricate, irradiate and analyze lithium ceramic materials for fusion blanket.

RECENT TECHNICAL HIGHLIGHTS

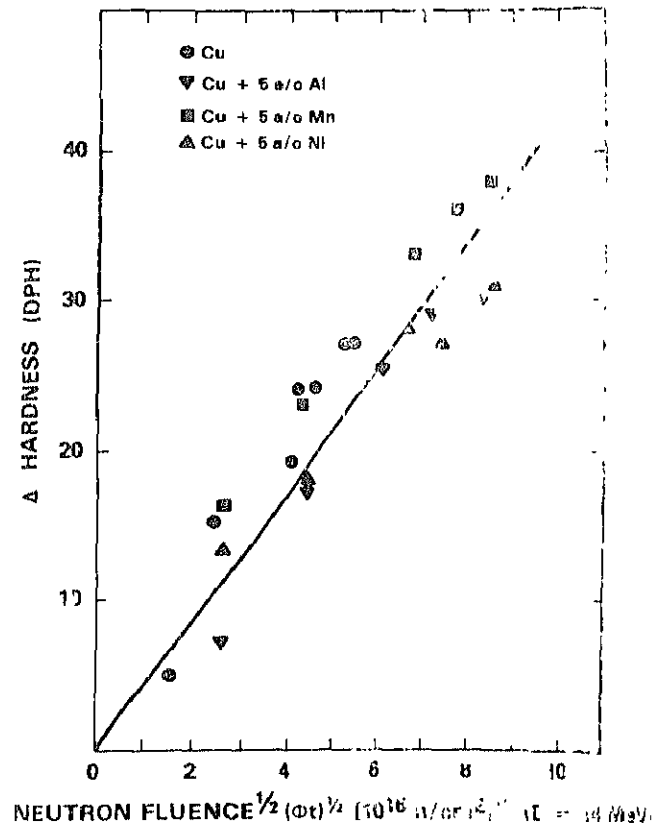
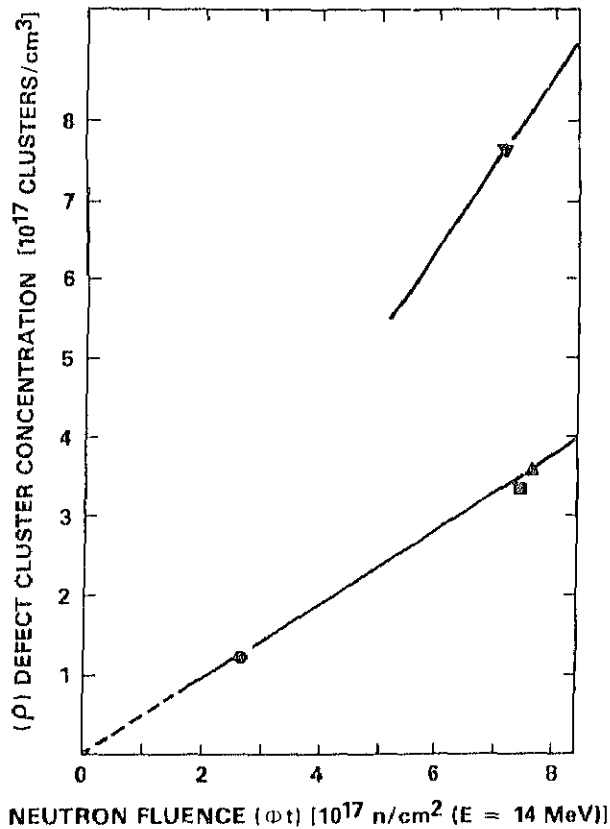
Technical feasibility for a Fusion Breeder Materials Irradiation experiment in EBR-II was been established.

EXPECTED NEAR-TERM ACCOMPLISHMENTS

Complete qualification of lithium ceramic materials for irradiation in EBR-II. Complete test design for EBR-II Solid Breeder Materials Test.

ALLOY INDEPENDENT IRRADIATION HARDENING INDICATES THAT ALLOY ADDITIONS INFLUENCE VISIBILITY OF DEFECT CLUSTERS

(14 MeV NEUTRONS; 25°C IRRADIATION TEMP.)



IRRADIATION EFFECTS ANALYSIS

HEDL

Cost Account - AKJ

Oper. X Inv.

Manager: D.G. Doran

Sub-Department: Technology

FY-81: \$ 700K

Customer:

Organization: DOE/OFE

Contact: K M. Zwilsky

Program: Fusion Reactor Materials
AT-15-30-34

OBJECTIVE

Clarify fundamental processes controlling material response to irradiation and develop correlations for extrapolation of fission-generated data base to fusion environments.

SCOPE

Analyze basic radiation damage events, model mechanisms of material response, and conduct experimental studies involving comparative fission and fusion irradiations

RECENT TECHNICAL HIGHLIGHTS

High resolution electron microscopy and recently developed microhardness measurement techniques were used to analyze the initiation of microstructural damage in specimens irradiated by fusion energy neutrons. A computer graphics capability was developed for analyzing atomistic damage models.

EXPECTED NEAR-TERM ACCOMPLISHMENTS

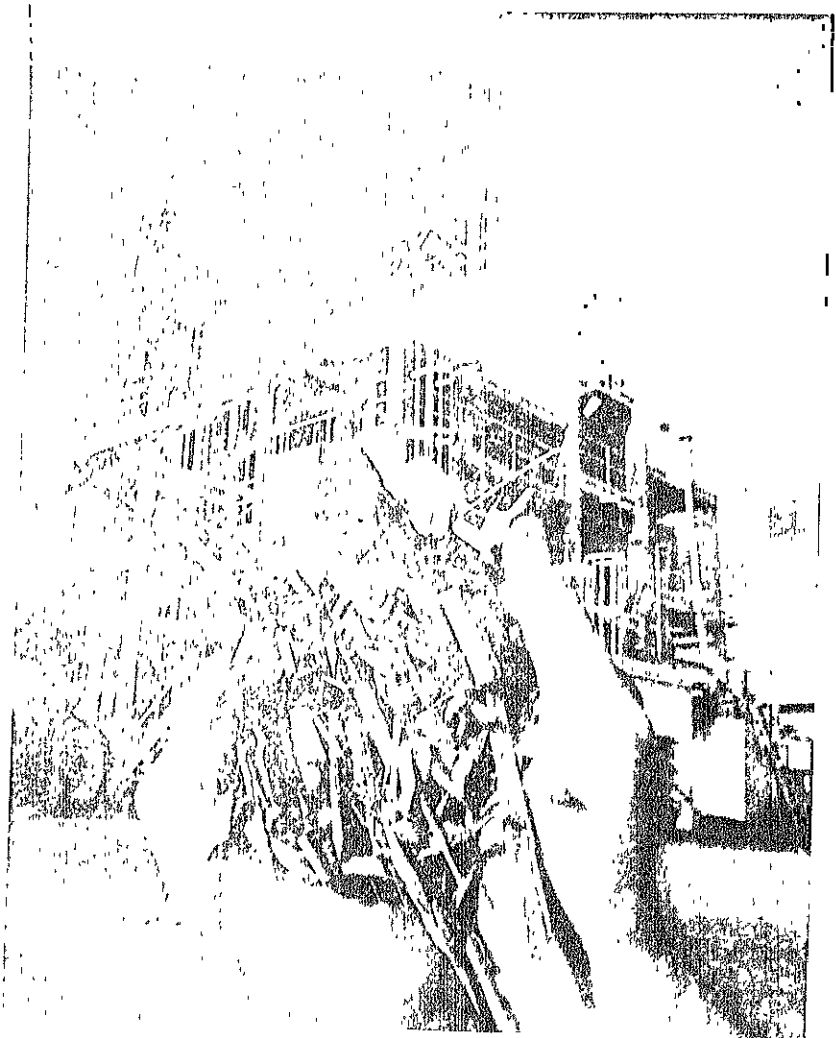
Atomistic modeling of high energy neutron damage production will be extended to higher energy events (several hundred KeV) and computer graphical analyses made. Electron microscopy will begin on specimens irradiated in Oak Ridge Research Reactor and EBR-II to determine helium effects on microstructure.

COAL MINING



BIOMASS DEVELOPMENT

Evaluation of
pollution impact
from non-point
sources.



OTI WATER RESOURCE STUDIES

HEDL

Cost Account - ALR

Oper. X Inv.

Manager: J.F. Fletcher
Sub-Department: Technology

FY-81: \$ 25K

Customer:

Organization: DOE/DEA/RID

Contact: F H Osterhault

Program: Energy Assessment Regional
Impacts Program
HA-01-03-04

OBJECTIVE

Provide water resource data and analytical support for Energy Assessment Regional Impact Programs.

SCOPE

Evaluate the nature and extent of non point pollution impacts expected from energy activities.

RECENT TECHNICAL HIGHLIGHTS

Requested evaluations and water resource data were submitted

EXPECTED NEAR-TERM ACCOMPLISHMENTS

Evaluation of pollution impacts associated with mining, synfuels development and biomass development will be completed.

IRFM FLOW MODEL FOR THERMAL HYDRAULIC TESTING OF REACTOR FLOW SYSTEMS

CRBRP REACTOR SYSTEMS THERMAL HYDRAULIC TESTING

HEDL
Cost Account - EBA
Oper. X Inv.

Manager: W.L. Thorne
Sub-Department: Technology

Customer:
Organization: CRBRP
Contact:
Program: Special Request

FY-81: \$ 890K

OBJECTIVE

Provide thermal hydraulic, vibration and mechanical testing of CRBRP reactor systems in accordance with Work Agreements L-274, L-294, and L-295

SCOPE

CRBRP Reactor Systems Testing at HEDL involves hydraulic and mechanical design verification test programs of reactor vessel internal systems including full-scale fuel and radial blanket assemblies, one-quarter scale inlet plenum and outlet plenum models, and other reactor vessel components under steady state reactor conditions

RECENT TECHNICAL HIGHLIGHTS

Completed data reduction and analysis of the original CRBRP Integral Reactor Flow Model (IRFM) Bypass Thermal Striping tests.

Performed initial CRBRP Duct Bending/Load Pad Strength testing.

Started flow and vibration testing on full scale prototypic CRBRP radial blanket fuel assembly.

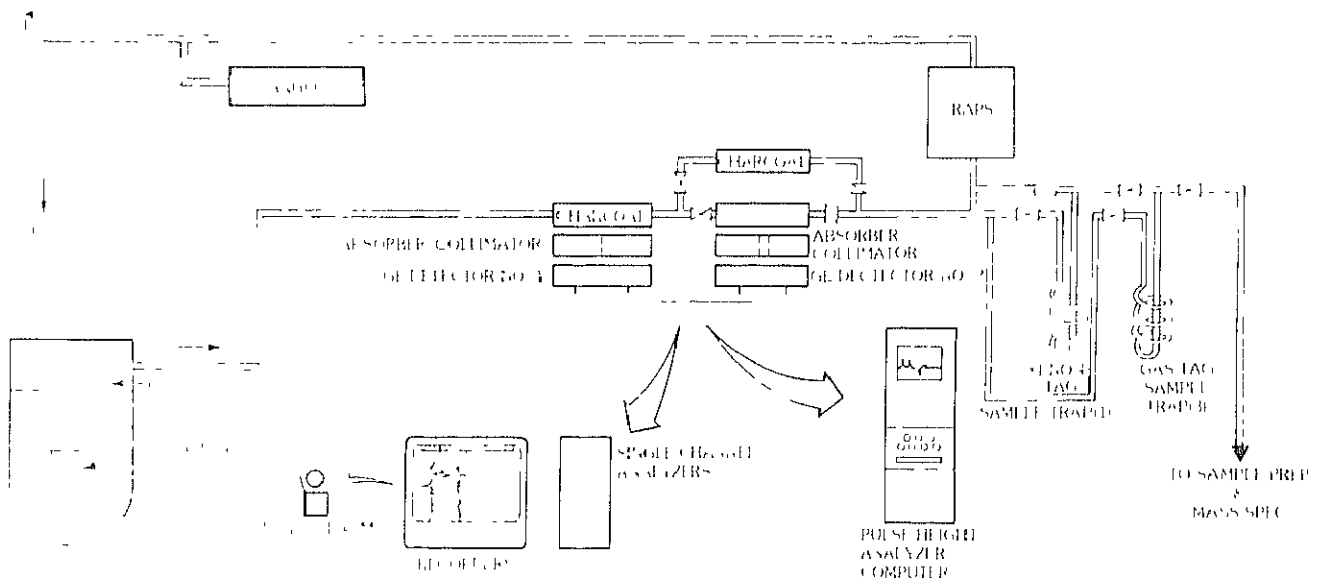
EXPECTED NEAR-TERM ACCOMPLISHMENTS

Perform Phase V of the Fuel Assembly Cavitation and Orifice Calibration tests and issue report.

Perform CRBRP Piston Ring Leakage test program and issue report.

Complete CRBRP Duct Bending/Load Pad Strength test program and issue report

COVER GAS MONITORING FOR CRBRP - PRELIMINARY DESIGN



CRBRP FUEL
FAILURE MONITORING

HEDL
Cost Account - EBC
Oper. X Inv.

Manager: J.J. McCown
Sub-Department: Technology

Customer:
Organization: CRBRP
Contact:
Program: Special Request

FY-81: \$ 90K

OBJECTIVE

Finalize basic parameters of cover gas Fuel Failure Monitoring sampling and analysis subsystem. Provide engineering consultation services during preliminary design.

SCOPE

Modify Prooftest Experiment, EX-154, at EBR-II and test suitability of modifications
Develop computer codes that use mass spectrometric and radiometric analyses to locate and characterize fuel failures.

RECENT TECHNICAL HIGHLIGHTS

Installed HEDL designed thick absorber in position #5 of Detector No. 2's device to improve very high countrate capability.

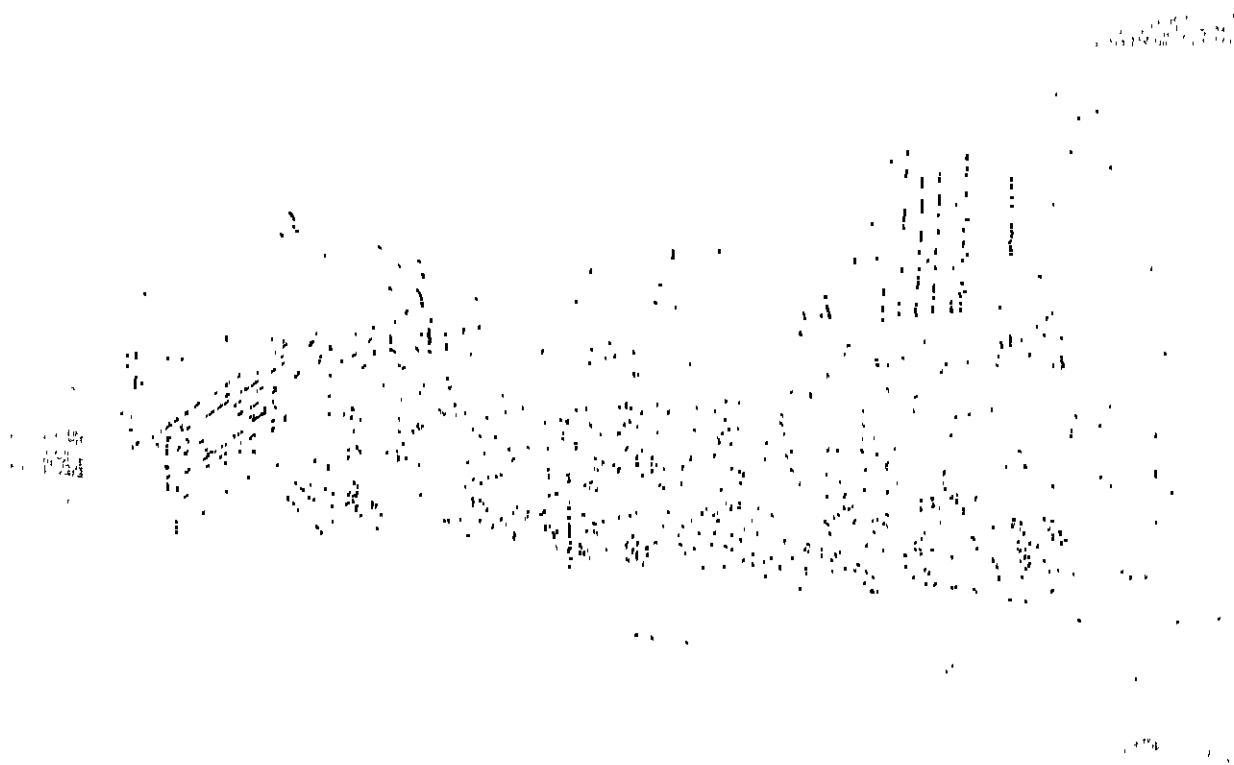
Tested modified Gamma Ray Subtract (GRS) software for use with compound absorber-collimators and with thick absorber.

EXPECTED NEAR-TERM ACCOMPLISHMENTS

Data on suitability of compound absorber-collimators and of thick absorbers will be provided.

The feasibility of using one rather than two sampling and measurement modules will be determined.

Modification and testing of GRS software will be completed.



TRANSPORTATION OF SPENT FUEL IN LWR SHIPPING CASK

SHIPPING CASK ANALYSIS

HEDL

Cost Account - EAA

Oper. X Inv.

Manager: J.F. Fletcher
Sub-Department: Technology

FY-81: \$ 120K

Customer:
Organization: NRC
Contact:
Program: Special Request

OBJECTIVE

Assist in standards development and compliance evaluations for NRC by developing a computer simulation model of the mechanical response of radioactive material shipping packages.

SCOPE

Develop computerized simulation model of mechanical responses of LWR spent fuel shipping casks in normal transport situations. Validate and parametrically extend the model; provide calculated results to apply to standards development.

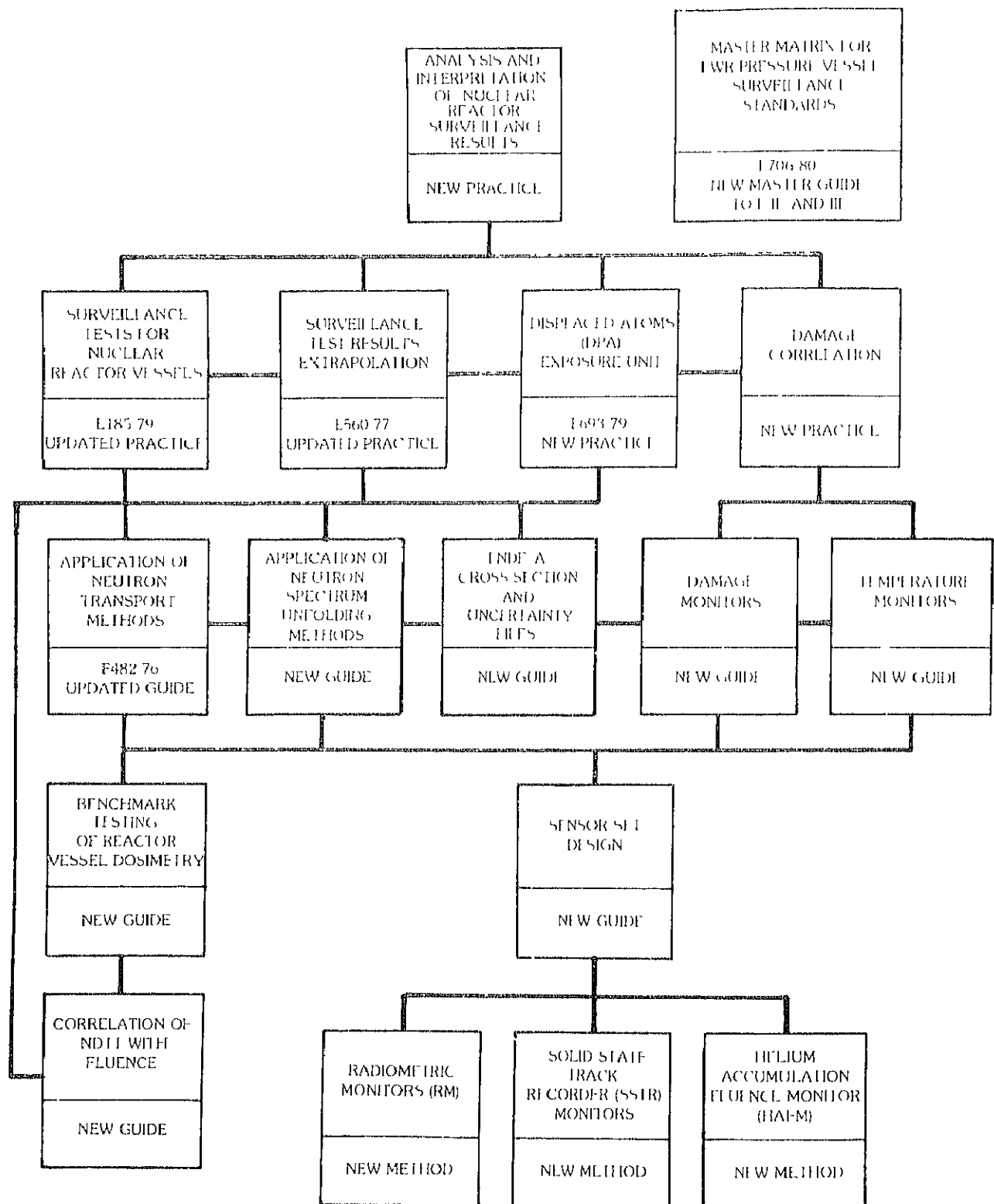
RECENT TECHNICAL HIGHLIGHTS

Preliminary validation of the computerized simulation model was completed. Parametric and sensitivity analysis were completed.

EXPECTED NEAR-TERM ACCOMPLISHMENTS

Complete model validation. Extend parametric and sensitivity analysis and supply data and methodology for purposes of developing regulatory guidelines.

ASTM STANDARDS FOR SURVEILLANCE OF NUCLEAR REACTOR PRESSURE VESSELS



LWR NEUTRON DOSIMETRY

HEDL

Cost Account - EAB

Oper. X **Inv.**

Manager: W N McElroy
Sub-Department: Technology

FY-81: \$ 588K

Customer:
Organization: NRC
Contact:
Program: Special Request

OBJECTIVE

Establish updated and improved ASTM standards for LWR pressure vessel irradiation surveillance, dosimetry, damage correlation, and associated reactor analysis and interpretation procedures

SCOPE

Prepare and write 17 ASTM recommended standards.

Perform supporting analytical and experimental work: validation and calibration of the recommended ASTM standards using "Standard, Reference, and Controlled Environment Benchmark Neutron Fields," Reactor "Test Regions," and Operating Power Reactor "Surveillance Positions."

RECENT TECHNICAL HIGHLIGHTS

Major physics and dosimetry studies in a low-flux level pressure vessel mockup at ORNL, including a "Blind Test" validation of physics calculations involving US and foreign participants, was completed.

A two-year metallurgical irradiation in a high-flux level pressure vessel mockup at ORNL was started. A shorter irradiation of a mockup surveillance capsule was completed.

EXPECTED NEAR-TERM ACCOMPLISHMENTS

Preparation and writing of key ASTM Practice, 1A will be completed.

Analysis of samples from the metallurgical irradiation at ORNL will be started. This will validate the accuracy of using surveillance capsule data to make end-of-life predictions for the pressure vessels of LWR operating power reactors.

LOFT ADVANCED FUEL ROD
INSTRUMENTATION DEVELOPMENT

HEDL
Cost Account - EAD
Oper. X Inv.

Manager: E.M. Sheen
Sub-Department: Reactor Core Supply
FY-81: \$ 164K

Customer:
Organization: NRC
Contact:
Program: Special Request

OBJECTIVE

Develop fuel rod instrumentation for NRC loss-of-flow tests (LOFT Program)

SCOPE

Develop fuel centerline temperature measurement systems to 2200°C fuel rod gas plenum pressure sensors, axial motion monitoring sensor and fast plenum thermocouples

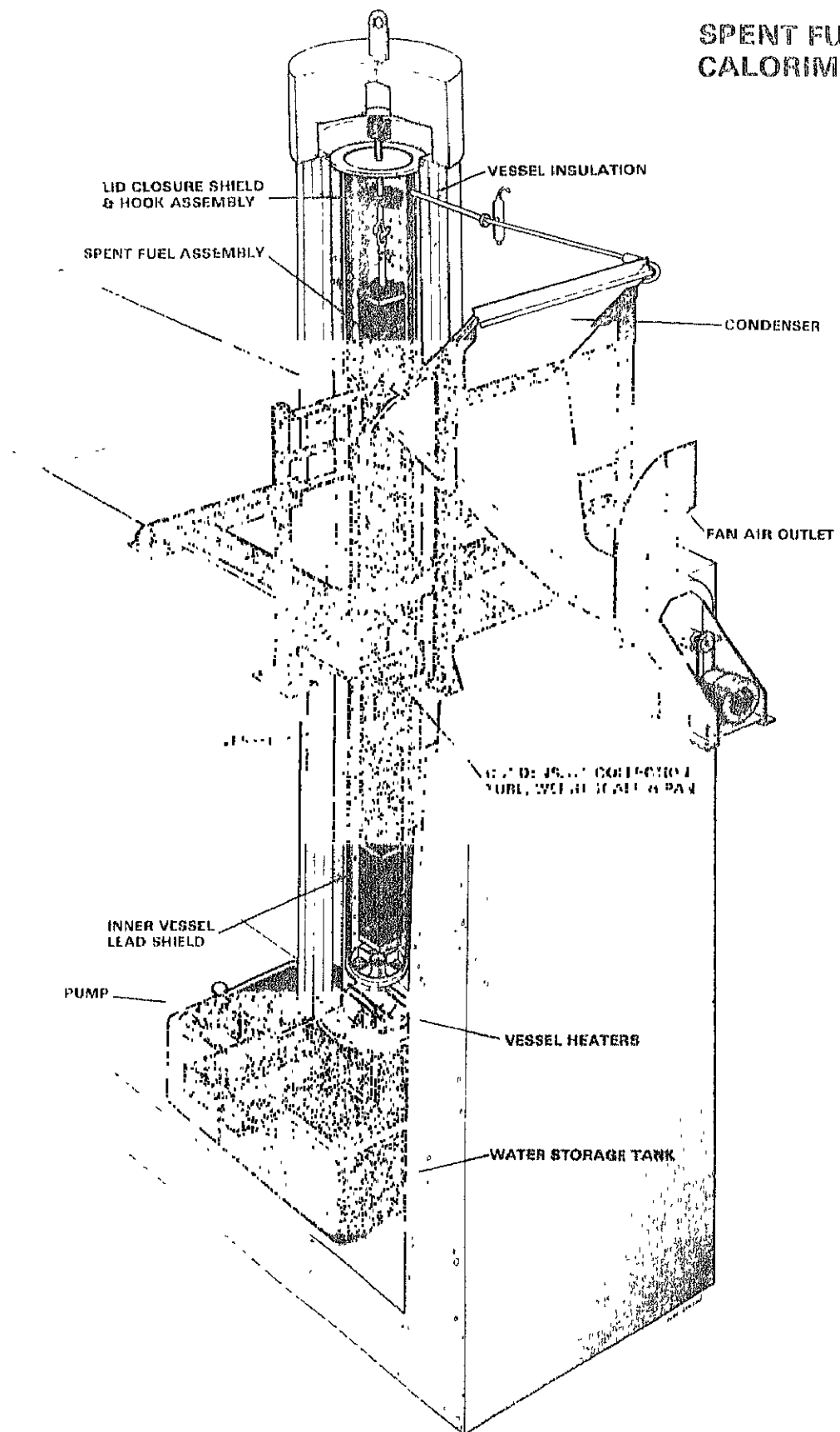
RECENT TECHNICAL HIGHLIGHTS

A fuel centerline temperature measurement system combining a Johnson Noise Power resistance coil and 2200°C thermocouple was tested in the laboratory. Vector signal analysis techniques improved fuel rod plenum pressure accuracy during temperature transients by a factor of 2.

EXPECTED NEAR-TERM ACCOMPLISHMENTS

A combined Johnson Noise Power and thermocouple for fuel centerline temperature measurement will be produced for trial in a test reactor.

SPENT FUEL CALORIMETER



NATIONAL WASTE TERMINAL STORAGE PROGRAM
SPENT FUEL ENGINEERING

HEDL
Cost Account - ECA
Oper. X Inv.

Manager: R.J. Cash
Sub-Department: Technology

FY-81: \$ 924K

Customer:
Organization: Office of Nuclear
Waste Isolation
Contact:
Program: Special Request

OBJECTIVE

Develop technology to select and characterize Unreprocessed Light Water Reactor spent fuel waste forms suitable for isolation in a mined geologic repository

SCOPE

Develop data bases and theoretical and empirical correlations to characterize and describe the behavior of spent fuel after geologic emplacement in mine repositories. Identify, test, select and qualify stabilizer materials for isolating spent fuel rods in geologic waste packages.

RECENT TECHNICAL HIGHLIGHTS

The first known decay heat measurement of a spent fuel assembly was successfully accomplished using a calorimeter designed, fabricated and installed at the Nevada Test Site by PNL and HEDL.

Results from initial elevated temperature whole rod tests show that significant stress relaxation occurs and stress rupture mechanisms can no longer be considered a primary mode for in-repository spent fuel breach.

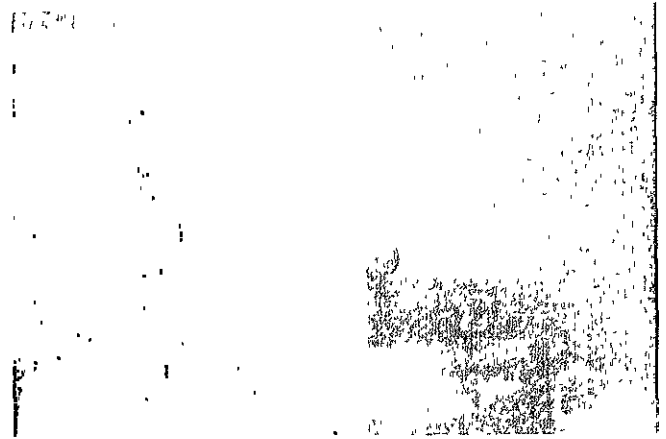
EXPECTED NEAR-TERM ACCOMPLISHMENTS

The Spent Fuel Engineering program at HEDL was redirected in FY-1981 to evaluate fuel waste form degradation mechanisms which affect long term resistance to release of radionuclides. This work will identify physical and chemical properties of spent fuel, theoretical and empirical correlations to bound degradation mechanisms and experimental work to provide information where a data base is lacking.

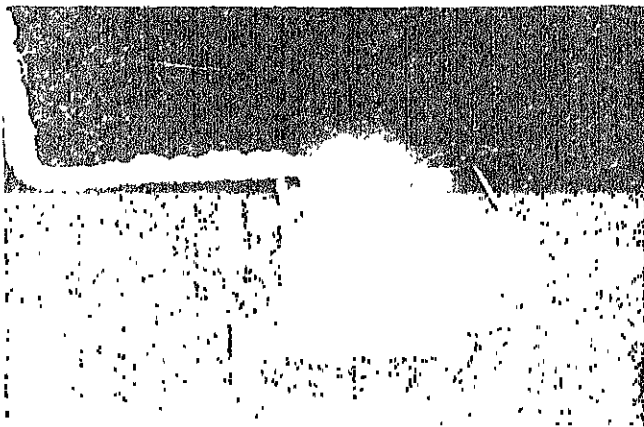
LITHIUM FIRE EXTINGUISHMENT



CARBON MICROSPHERES BY SPREADER



CARBONATE BASE POWDER BY SPREADER



CARBON MICROSPHERES BY SPRAYER



CARBON MICROSPHERES BY PRE-ADD

FUSION REACTOR SAFETY SUPPORT STUDIES

HEDL

Cost Account - ECB

Oper. X Inv.

Manager: L.D. Muhlestein
Sub-Department: Technology

FY-81: \$ 441K

Customer:
Organization: DOE/OFE
Contact:
Program: Special Request

OBJECTIVE

Provide experimental data regarding the use of liquid lithium and alternative breeding and coolant materials in fusion reactors, and maintain research facilities to support safety analysis and design

SCOPE

Complete lithium reaction scoping studies to include lithium-atmosphere, concrete and insulating material reactions.

Develop and proof test lithium-reaction extinguishment and control techniques.

Determine lithium reaction aerosol behavior and develop and proof test effluent control concepts.

Complete alternate coolant/blanket materials interaction scoping studies.

RECENT TECHNICAL HIGHLIGHTS

Completed lithium reaction scoping studies investigating reactions of lithium with various gaseous atmospheres, various types of concretes, and various insulating materials.

EXPECTED NEAR-TERM ACCOMPLISHMENTS

Complete tests to guide computer code development and experimentally demonstrate that temperatures and pressures which may result from lithium spray or pool reactions can be adequately predicted.

Complete scoping studies of alternative coolant and breeding materials to determine interactions and compatibility under postulated accident conditions.